

Characteristics in Water Phantom of Epithermal Neutron Beam Produced by Double Layer Beam Shaping Assembly

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KEYWORDS

Absorbed dose DLBSA Epithermal neutron & gamma distribution Water phantom ABSTRACT A double layer beam shaping assembly (DLBSA) was designed to produce epithermal neutrons for BNCT purposes. The Monte Carlo N-Particle eXtended program was used as the software to design the DLBSA and phantom. Distribution of epithermal neutron and gamma flux in the DLBSA and phantom and absorbed dose in the phantom were computed using the Particle and Heavy Ion Transport code System program. Testing results of epithermal neutron beam irradiation of the water phantom showed that epithermal neutrons were thermalized and penetrated the phantom up to a depth of 12 cm. The maximum value of the absorbed dose was 2×10^{-3} Gy at a depth of 2 cm in the phantom.

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1. INTRODUCTION

Boron neutron capture therapy (BNCT) is a method in cancer therapy that causes minimal damage to normal tissues. This method makes use of the ability of boron to capture the neutron beam, subsequently killing cancer cells (Sauerwein et al. 2012).

In BNCT, highly lethal energetic particles (α , ⁷Li) produced after a breakup process of ¹¹B nuclei when ¹⁰B atom captures a thermal neutron are used. The decay process of the boron–neutron reaction is shown in Equation 1 (Moghaddasi and Bezak 2018):

$${}^{1}n + {}^{10}B \rightarrow {}^{11}B + \gamma$$

$${}^{1}n + {}^{10}B \rightarrow {}^{7}Li + {}^{4}He + \gamma$$

$${}^{1}n + {}^{10}B \rightarrow {}^{7}Li + {}^{4}He$$
(1)

The products of this reaction have high linear energy transfer characteristics (α particle approximately 150 keV μ m⁻¹, ⁷Li-nucleus approximately 175 keV μ m⁻¹). The path lengths of these particles in tissues are in the range of 4.5–10 μ m: hence resulting in an energy deposition limited to the diameter of a single cell.

Nuclear reactor neutron sources have long been used for BNCT, because high intensity neutron beams have been supplied only by reactors. However, many of these reactors have been shut down, reducing the number of reactors available for BNCT. Many reactors have been constructed only recently for BNCT. Conversely, accelerator-based neutron sources are becoming popular in neutron application fields (Kiyanagi 2018). Accelerators have several potential advantages related to safety, cost, and high neutron flux in the keV range, compared with reactor-based neutron sources for clinical radiotherapy (Peng et al. 2019). One of accelerators used in BNCT comes from a cyclotron (Hashimoto et al. 2014). The resulting neutrons from the cyclotron are regulated so as to comply with the requirement set by the International Atomic Energy Agency (IAEA). Based on these criteria, the minimum beam intensity of the epithermal neutrons (1 eV < E < 10 keV) should be $5 \times 10^8 \text{ n/cm}^2$.s, the number for fast neutron (E > 10 keV) dose rate per epithermal neutrons flux and gamma dose rate per epithermal neutron flux should be less than 2×10^{-13} Gy cm² and the minimum number for the ratio of the epithermal neutrons flux to the thermal neutrons (E < 1 eV) flux should be 100 (Ganjeh and Eslami-Kalantari 2019). The part of the cyclotron that is capable of processing the neutron beam is a beam shaping assembly (BSA).

Until now, most efforts for cyclotron-based BNCT have been focused on the design of the BSA to investigate the feasibility of clinical neutron beams having the desired characteristics for patient irradiation. To achieve this, many types of BSA have been designed using Monte Carlo N-Particle Transport Code (MCNP) and the Particle and Heavy Ion Transport code System (PHITS) program (Pelowitz 2008; Sato et al. 2013). The design is expected to produce neutrons that meet the IAEA standard of quality.

BSA designs used in neutron sources typically consist of the moderator, filter, reflector, and collimator as their main components (Kasesaz et al. 2014). Each of the components is commonly designed with a single layer configuration, i.e. they only use one type of material. Such a single-layer configuration has a weakness in that the components of BSA are not maximal in processing neutron radiation beams; hence the result is normally not optimal. To overcome this weakness, double layer and even multilayer configurations have been developed. Such configurations are yet to be optimized, to obtain better radiation beams, and tested, to find if the characteristics of the resulting beams meet the requirements for BNCT (Monshizadeh et al. 2015; Tanaka et al. 2011; Kasesaz et al. 2013).

In principle there are two ways to find the quality of beams for BNCT, either by assessing their quality in the air or in a water phantom. Assessment of radiation beams in the air complies with the IAEA standard (International Atomic Energy Agency 2001). As for the assessment in a water phantom, the emphasis is on the ability of radiation beams to penetrate the phantom and the dose of neutron sustained by a tumor (Ghal–Eh et al. 2017). A water phantom is typically chosen as the testing material because 70% of the human body consists of water (Tsukamoto et al. 2011). This article reports the characteristics of neutron and gamma distribution in a DLBSA and phantom, as well as the absorbed dose in a water phantom.

2. MATERIALS AND METHODS

The proton source was modeled as 30 MeV protons impinging on ⁹Be target with a diameter and thickness of 5 cm and 0.5 cm, respectively. High-energy neutrons are supposed to originate from ⁹Be(p,n) reactions (Hashimoto et al. 2014). They are subsequently processed using a double layer beam shaping assembly (DLBSA) to yield epithermal neutrons. The configuration of the intended DLBSA is shown in Figure 1, and a three-dimensional model is shown in Figure 2.

The materials used as the moderator in the design of DLBSA were aluminum (Al) and LiF. The reason for the selection of Al was because of its high scattering cross-section. Aluminum has a high cross-section at energies above 10 keV (Zaidi et al. 2017).

The materials used as reflectors were Pb and FeC, which have a high density and ability to scatter fast neutron extremely well (Sato et al. 2013). Graphite (C) was also used as a reflector for its low cost. Apart from being cheap it also has a high scattering cross-section and low absorption, particularly at energies above 1 MeV (Türkmen et al. 2017).

The collimator component under consideration was made of Ni and borated polyethylene materials. Ni is considered to be a stable element when it interacts with neutrons.

For a fast neutron filter, Fe was used. The effectiveness of Fe as a high energy neutron filter owes to its ability to inelastically scatter high energy neutrons passing through it. Fe is deemed to be superior in filtering fast neutrons. The ability of Fe to filter fast neutrons derives from its resonant cross-section, which is above 10 keV (Asnal et al. 2015).

Thermal neutrons are filtered using a material with a high atomic number. Among the atoms with a high thermal neutron absorption cross-section is Cd. Cd is frequently used as a thermal neutron filter. A cross-section of 20,600 barn is reasonably effective to absorb thermal neutrons (Os-awa et al. 2017; Asnal et al. 2015).

The material used for shielding was Pb. It has a relatively constant attenuation coefficient, i.e. $0.05 \text{ cm}^2/\text{g}$ to be able to absorb gamma rays with energies of 1-10 MeV (Türkmen et al. 2017).

Epithermal neutrons leaving the DLBSA were subsequently imposed on the water phantom, shaped as a round ball, which was placed 1 cm at the front of the DLBSA. The composition of the water phantom in this study was 11.2% of H atoms and 88.8% of O atoms, with a density of 1000 kg/cm³ (Raaijmakers et al. 2000).

Monte Carlo simulation was carried out using the MC-NPX and PHITS programs. The MNCPX program was used



FIGURE 1. Configuration of DLBSA and water phantom.



FIGURE 2. Three-dimensional model of DLBSA and water phantom.

as the software to design the DLBSA and phantom (Pelowitz 2008). The distributions of epithermal neutron and gamma flux in the DLBSA and phantom and neutron absorbed dose in the phantom were computed using the PHITS program (Sato et al. 2013). The track and deposit tally were used in the PHITS calculation. To draw the particle track and visualization geometry of DLBSA, the ANGEL software was used. The transport was based on the cross-section data library JENDL-4.0 for neutrons and photons, and intra-nuclear cascade (INCL4.6) for protons.

3. RESULTS AND DISCUSSION

3.1 Characteristics of neutron and gamma beams in DLBSA and water phantom

Figure 3 shows the distribution of epithermal neutrons in the DLBSA. The fast neutrons mainly resulted from reactions of 30 MeV protons with ⁹Be target material through ${}^{9}_{4}Be(p,n){}^{9}_{5}B$. Fast neutrons interacted with Al and LiF moderator, producing epithermal neutrons. The flux of epithermal neutrons around the moderator and filter reached 10¹¹ n/cm².s. The increase in the amount of epithermal neutrons was also sustained by the presence of Fe placed in front of the moderator filtering high energy neutrons. The effectiveness of Fe as a filter for high energy neutrons is due to its ability to inelastically scatter high energy neutrons passing through the Fe material (Asnal et al. 2015). The flux of epithermal neutrons computed at the end of the collimator exceeded 10^{9} n/cm².s.

An epithermal neutron flux entering the phantom continually decreases in energy (as reflected in the change in color from yellow to blue in the phantom). The decrease in epithermal neutron flux is due to epithermal neutrons transforming into thermal neutrons during interactions with hydrogen atoms. This process is called thermalization (Mishima 1996). The neutron flux in the phantom decreased from 10^9 n/cm^2 .s to 10^6 n/cm^2 .s.



FIGURE 3. Distribution of epithermal neutron flux in DLBSA and water phantom.



FIGURE 4. Distribution of gamma in DLBSA and water phantom.

Figure 4 shows the distribution of gamma particles in the DLBSA and water phantom. The gamma particle flux around the beryllium target was $10^{12} \gamma/s.cm^2$. The particles were dominantly produced from interactions of protons with a beryllium target through ${}^{9}_{4}Be(p,\alpha){}^{6}_{3}Li^{*}(\gamma){}^{6}_{3}Li$ reactions. A small fraction of gamma rays was also generated from capture reactions through ${}^{9}_{4}Be(p,\gamma){}^{10}_{4}Be$ reactions and inelastic collision mechanisms in the form of ${}^{9}_{4}Be(n,n'\gamma)$ reaction (Hu et al. 2016). Gamma particles were produced from the reaction of neutrons with aluminum through ${}^{27}Al(n,\gamma){}^{28}Al$ reactions (Ma et al. 2015). Gamma particles enter the phantom and interact with H and O, losing their energy through mechanisms of photoelectric effect, Compton scattering, and pair production (Lamarsh and Baratta 2001). These interactions cause gamma ray flux to continually decline in the phantom.

3.2 Neutron absorbed dose in water phantom

Figure 5 shows the neutron absorbed dose in the water phantom, the maximum value of which was 2×10^{-3} Gy, obtained at a depth of 2 cm from the surface of the phantom. This value is in accordance with the work of Tanaka et al. (2011) and Morcos and Naguib (2012). The deeper the epithermal neutron penetrates the phantom, the greater the increase in thermalization, causing the value of neutron flux to diminish. The decrease in neutron flux is caused by the thermalization of neutrons with H (Mishima 1996).



FIGURE 5. Neutron absorbed dose characteristic in water phantom.

Based on the neutron characteristics in the phantom, the neutron beams produced by the DLBSA can be considered as a neutron source for BNCT. The ability of epithermal neutrons in a phantom shows that the neutron beams from the DLBSA can be utilized as a neutron source for the treatment of cancerous tumors situated at 2-8 cm. Some of the types of cancers that can be treated using such a neutron source are head and neck cancer, glioblastoma, lung cancer, breast cancer, pancreatic cancer, brain tumors and sarcoma (Moss 2014; Mirzaei et al. 2016).

4. CONCLUSIONS

A double layer beam shaping assembly was designed to produce epithermal neutrons for BNCT purposes. The results of modeling of the DLBSA and phantom show that the distribution of epithermal neutrons in the DLBSA and phantom continually decrease in neutron and gamma flux. The decrease in epithermal neutron flux and gamma is due to the thermalization process. Epithermal neutrons penetrated the phantom up to 12 cm in depth. Meanwhile, the maximum neutron absorbed dose in the phantom was found at a depth of 2 cm, with 2×10^{-3} Gy in value. These results show that the neutron beams produced by the DLBSA are adequate as a neutron source for BNCT, particularly in the treatment of deep-seated tumors.

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